Mr. A. Christopher Bakken III, Senior Vice President and Chief Nuclear Officer Indiana Michigan Power Company Nuclear Generation Group 500 Circle Drive Buchanan, MI 49107

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNIT 2 - ISSUANCE OF AMENDMENT

REGARDING MEASUREMENT UNCERTAINTY POWER UPRATE

(TAC NO. MB6751)

Dear Mr. Bakken:

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 259 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit 2. The amendment consists of changes to the Technical Specifications in response to your application dated November 15, 2002, as supplemented February 24 and April 25, 2003.

The amendment increases the licensed reactor core power level by 1.66 percent from 3411 megawatts thermal (MWt) to 3468 MWt. The power level increase is considered a measurement uncertainty recapture power uprate.

By letter dated April 18, 2003, we issued a draft version of the enclosed safety evaluation (SE) and requested that you review it to verify that factual information is accurate and complete. By your April 25, 2003, supplemental letter, you provided comments on the draft SE. We have evaluated your comments and have incorporated them as appropriate. We note that your comments did not change our findings or conclusions discussed in the draft SE.

A copy of our related SE is enclosed. A Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

John F. Stang, Senior Project Manager, Section 1 Project Directorate III Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket No. 50-316

Enclosures: 1. Amendment No. 259 to DPR-74

2. Safety Evaluation

cc w/encls: See next page

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ADAMS Accession No. ML030990094 (Cover Letter and Amendment)

ADAMS Accession No. ML031260667 (License and TS Pages)

ADAMS Accession No. ML030990129 (Safety Evaluation)

ADAMS Accession No. ML030990132 (Package)

CC:

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INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 259 License No. DPR-74

- 1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated November 15, 2002, as supplemented February 24 and April 25, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and
 (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 259, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

In addition, the license is amended to revise paragraph 2.C.(1) to reflect the increase in the reactor core power level. Paragraph 2.C.(1) is hereby amended to read as follows:

The licensee is authorized to operate the Donald C. Cook Nuclear Plant, Unit No. 2, at steady state reactor core power levels not to exceed 3468 megawatts (thermal).

- 3. This license amendment is effective as of the date of issuance and shall be implemented within 90 days of the date of issuance. Prior to implementation of the license amendment, the licensee shall:
 - A. Install a leading edge flowmeter CheckPlus system. The design change for the installation will include instrumentation rescaling, Updated Final Safety Analysis Report (UFSAR) revision, maintenance and operational procedure revisions, training, monitoring iso-phase bus duct temperature, and implementation of the LEFM CheckPlus system out-of-service administrative technical requirements, as described in the licensee's November 15, 2002, application. The UFSAR revision shall be reflected in the next update of the UFSAR submitted to the NRC pursuant to 10 CFR 50.71(e)
 - B. Complete a formal reload safety evaluation, as described in the licensee's November 15, 2002, application, and evaluated in the associated safety evaluation by the Office of Nuclear Reactor Regulation, dated May 2, 2003. The evaluation shall be reflected in the next update of the UFSAR submitted to the NRC pursuant to 10 CFR 50.71(e).
 - C. Complete an analysis of the steam dump valves flow capacity at the uprated power level and based on this analysis, implement any changes/adjustments necessary to ensure the valves have sufficient capacity prior to implementing the 1.66-percent power uprate. The analysis and any changes/adjustments to the steam dump valves shall be reflected in the next update of the UFSAR submitted to the NRC pursuant to 10 CFR 50.71(e).

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Ledyard B. Marsh, Acting Director Division of Licensing Project Management Office of Nuclear Reactor Regulation

Attachment: Changes to the Operating License and Technical Specifications

Date of Issuance: May 2, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 259

FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO. 50-316

Replace the following page of Facility Operating License No. DPR-74 with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

<u>REMOVE</u>	<u>INSERT</u>		
3	3		

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE	<u>INSERT</u>		
1-1	1-1		
3/4 5-3	3/4 5-3		
3/4 7-2	3/4 7-2		